ECRH Plasma Experiments at B=1.0T in HSX

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18th International Toki Conference – Development of the Physics and Technology of Stellarators/Heliotrons en route to DEMO

Outline



• U.S. Program Evolution – Strategic planning process

 First use of 3D equilibrium reconstruction Code V3FIT to model data in stellarator

-First observation of helical Pfirsch-Schlüter current; good agreement with measured bootstrap current

Progress on neoclassical and turbulent transport modeling

- Neoclassical: PENTA code includes momentum conservation and parallel flow \rightarrow predicts lower E_r than standard ambipolarity constraint - Turbulent: GS2, a 3D gyrokinetic code \rightarrow good agreement with

experimental T_e profile outside core as well as confinement scaling

First observation of internal transport barrier (CERC mode) in quasisymmetric stellarator

– Close proximity of electron and ion roots in core \rightarrow ExB shear suppression of turbulence results in very peaked T_e profile

Future Directions for HSX

US Formulating a New Fusion Strategic Plan

• A Research Needs Workshop (ReNeW) will be held June 7-13, 2009 to provide the US DoE with a series of community-developed initiatives for guidance in MFE research

• DoE will utilize the results to formulate a vision and specific goals for MFE research activities over the next 15-20 year timeframe

- International coordination specifically requested

• Efforts structured around recent "Greenwald Report": "Priorities, Gaps and Opportunities: Towards a Long-Range Strategic Plan for Magnetic Fusion Energy", with five themes:

-Achieving and Understanding the Burning Plasma State

- -Creating Predictable High Performance Steady State Plasmas
- -Taming the Plasma Material Interface
- -Harnessing Fusion Power
- -Optimizing the Magnetic Configuration

Website for information: <u>http://burningplasma.org/renew.html</u>

US Stellarator Program Depends Critically on ReNeW Process

•The US FESAC "Toroidal Alternates Panel" has recently issued a report to DoE on the goals, scientific/technical questions, and gaps and opportunities non-tokamak toroidal systems 'in the ITER-era'

HSX

http://fusion.gat.com/tap/final_report.php/

•The report states: "The US stellarator program remains committed to the development of the quasi-symmetric stellarator approach"

- -Strong connection to 2-D tokamak physics
- -Reduced transport and good energetic particle confinement
- -Low flow damping and importance of plasma flows

•There is an understanding that a 'significant scale' device will be needed in the ITER-era

•Initiatives to address this need should be a result of the ReNeW process with strong community consent

Stellarator panel being formed: D. Anderson, J. Harris, C. Hegna, S. Knowlton, P. Politzer, A. Rieman, A. Ware, H. Weitzner

Quasihelical Stellarators have large effective transform



HSX is a Quasihelically Symmetric (QHS) Stellarator:

toroidal stellarator with *almost no* toroidal curvature $\rightarrow \epsilon_t = 0.0025$ in aspect ratio 8 device

Near symmetry in |B| :

 $B = B_0[1 - \varepsilon_h \cos (N\phi - m\theta)]$

In straight field line coordinates $\theta = \mathbf{i} \phi$ so,

 $\mathsf{B} = \mathsf{B}_0[1 - \varepsilon_h \cos(\mathsf{N} - \mathsf{m}_{\mathfrak{t}})\phi]$

In HSX N = 4, m = 1 and $t \ge 1$

 $t_{eff} = N - m t \sim 3$

Neoclassical currents reduced by this factor



V3FIT⁺ code calculates magnetic flux at pick-up coils due to neoclassical currents



• V3FIT: Equilibrium reconstruction for 3D toroidal devices, similar to EFIT

• Reconstruction is goal for CTH stellarator at Auburn University

 Applicable to tokamak with nonaxisymmetric magnetic fields: edge ripple and field errors, RMP's for ELM suppression, inhibit onset of NTM, generate plasma rotation, 3D shaping for external transform

• HSX: compare V3FIT calculation to pick-up coil data \rightarrow bootstrap current as function of E_r and symmetry-breaking, as well as Pfirsch-Schlüter current



⁺Hirshman, Lazarus, Hanson, Knowlton, Lao, PoP 11, 595 (2004)

Helical Pfirsch-Schlüter current demonstrated by phase shift of B_r measurements separated by ~1/3 field period

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16 3-axis pick up coils mounted in a poloidal array

Bootstrap current characterized by increasing B_{θ} offset with time



• Bootstrap current increases with time while density and stored energy remain constant

 V3FIT calculation for t = 50 ms and steady-state

• Bootstrap current is opposite direction and reduced by $n - m_{\tilde{t}} \sim 3$ compared to tokamak, as predicted

Quasisymmetry can be degraded with auxiliary coils

- Auxiliary coils add n=4 and 8, m=0 terms to the magnetic field spectrum
 - Called the Mirror configuration as compared to QHS
 - Increases neoclassical transport, flow damping similar to conventional stellarator



Effective ripple at r/a ~
2/3 increases from 0.005 to 0.04

- Little change in volume, transform and well depth
- Towards axis, $\epsilon_{\rm eff}$ for conventional stellarator can approach QHS

Need ~ 70% more ECRH power in Mirror for similar $\rm T_e$ profile in QHS

HS)



- Adjust power to get similar profiles 26 kW in QHS, 44 kW in Mirror
 - $\tau_{\rm E}$ = 4 ms for QHS, 3 ms Mirror

 Compare anomalous transport without assumptions as to scaling of temperature, density and gradients

 Theory (Shaing, Sugama & Watanabe, Mynick & Boozer) and expts in LHD suggest reducing neoclassical transport may also reduce anomalous transport – Is there any evidence for this in HSX?



- PENTA code (Spong 2005) includes momentum conservation and parallel flows (based on Sugama & Nishimura 2002) compared to DKES calculation
- E_r for QHS electron root from PENTA ~ 1/2 DKES using standard ambipolarity
- Agreement much better for Mirror, characteristic of conventional stellarator
- E_r measurements based on CHERS are forthcoming



- Possibility that anomalous transport lower for QHS in core where $\rm T_e$ is very peaked but need

- experimental measurement of E_r to verify neoclassical calculation
- nonlinear gyrokinetic modeling of turbulent transport

First evidence of internal transport barrier in HSX



• Steep T_e gradient at core is first evidence of CERC – core electron root confinement – in a quasisymmetric stellarator at low ϵ_{eff}

• Linear growth rates due to TEM calculated by 3D gyrokinetic code GS2

• Single class of trapped particles in QHS allows simpler quasilinear Weiland model to compute anomalous thermal diffusivity

 Curvature in HSX ~ 3 times that in tokamak with same major radius → need to account for local geometry

• Close proximity of electron root to ion root in ECRH plasma leads to E x B shear stabilization of turbulence

Transport due to TEM overestimated at plasma core where electron/ion root transition occurs





- Inside plasma core, anomalous $\chi_{\!e}$ is factor 10-20 higher than experiment
- $\bullet~{\rm E_r}$ and ${\rm T_e}$ can be modeled with transport equations:
 - **D**_E is electric field diffusion coefficient

 $\mathbf{Q}_{\mathbf{e}}^{-}$ is heat flux due to sum of anomalous and neoclassical

$$\frac{\partial E_{r}}{\partial t} - \frac{\partial}{\partial V} \left[\left\langle \nabla V \right\rangle D_{E} \left(\frac{\partial E_{r}}{\partial r} - \frac{E_{r}}{r} \right) \right] = \frac{e}{\epsilon_{\perp}} (\Gamma_{e} - \Gamma_{i})$$
$$\frac{3}{2} n_{e} \frac{\partial T_{e}}{\partial t} - \frac{\partial}{\partial V} \left[\left\langle \nabla V \right\rangle Q_{e} \right] = P_{ECRH} (\rho)$$

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Sharp gradient in T_e profile corresponds to shearing rate >> linear growth rate



 Shearing rate greater than maximum linear growth rate inside r/a ~ 0.3

• ExB shear suppresses turbulence: multiplying diffusivity by factor determined by quench rule:

 $\begin{array}{l} \text{max (1-}\alpha_{\text{E}}\gamma_{\text{E}}/\gamma_{\text{max}} \text{ ,0)} \\ \gamma_{\text{E}} = \text{shearing rate} \\ \gamma_{\text{max}} = \text{maximum growth rate} \end{array}$

• Without shear suppression ($\alpha_E = 0$), T_e at core is underestimated

• $\alpha_E = 0.3$ gives good agreement with temperature at core

Guttenfelder, et.al., PRL 101, 215002 (2008)

Future Directions in HSX



Quasisymmetry provides strong scientific connection between tokamaks and stellarators

- Identified by TAP panel as high priority item
- HSX provides a good opportunity in near-term
- Focus items:
- Conduct experiments on differences between QS and non-QS HSX plasmas
 - Explore whether lower neoclassical transport → lower anomalous
 - Measure E_r and plasma flow (CXRS) and compare to PENTA code
 - Determine time evolution and E_r dependence of bootstrap current
 - Heat ions and measure T_i distribution and confinement (new initiative)
 - Determine effect of nonresonant fields on plasma flow (new initiative w/DIIID)
- Advance understanding of possible connection of QS to improved turbulent transport
 - Determine relation of configuration to zonal flows and turbulent transport (GENE, GS2 codes) → compare to experiment when appropriate
 - Begin search for turbulent transport optimized configuration





- Comparison of V3FIT to experiment confirms helical Pfirsch-Schlüter current, also magnitude and direction of bootstrap current
 - Consistent with lack of toroidal curvature and high effective transform for a quasihelically symmetric stellarator
- PENTA calculation yields lower E_r for electron root solution when momentum conservation and parallel flows included
- Electron thermal diffusivity smaller in QHS than Mirror
- Anomalous transport model provides reasonable fit to temperature profile (outside core) and global energy confinement time
- First evidence of internal transport barrier (CERC mode) in a quasisymmetric stellarator
 - ExB suppression of turbulence needed to explain very peaked core T_e

Single class of trapped particles in HSX allows 2D tokamak model for anomalous transport calculations HS **Growth Rates** GS2 - HSX Weiland - HSX Weiland - Tokamak 6 6 6 a/L n_e gradient γ (10⁵ s⁻¹) 4 4 2 2 T_e gradient n 0 0 n 2 **N** 4 2 4 2 0 4 0 a/L_{Te} a/L_{Te} a/L_{Te}

• Simpler quasilinear 2D Weiland model validated by 3D linear gyrokinetic calculations using GS2 and exact geometry

• Curvature in HSX ~ 3 times that in tokamak with same major radius

 Strictly tokamak model underestimates growth rates → needs correction for HSX local geometry

Weiland model reproduces confinement scaling



• Captures scaling and magnitude of confinement times at B = 1.0 T

HSX

• Consistent with stellarator scaling ISS04:

 $\tau \thicksim P^{\text{-0.6}}$

• Without specific HSX geometry substitutions, predicted confinement time 2-3 times larger