Progress & Challenges of CW operation



LHD







The challenge: At reactor parameters (beta, n, H factor), demonstrate and optimize:

– Burning plasmas

- ITER after >2020
- Operation in CW or QCW (quasi CW, pulsed with high duty cycle)
 - LHD, TS, East, KSTAR, ST1, W7x + ...but no QCW Tokamaks foreseen; When ? rapid progress is essential!



Steady-state refers to a particular physics timescales

On present devices, it often refers to the Energy confinement time, sometimes to the current diffusion time



CW demonstration requires: t > largest time scales They are set by erosion, deposition, flake formation ~ several hours



• What counts is the engineering Q_{eng} $- Q_{eng} = P_{fus}/(P_{aux} + P_{inj}/\eta)$ $= \eta Q/(1 + \eta P_{aux}/P_{inj})$ with $Q = P_{fus}/P_{inj}$ ITER:

 η < 0.5; P_{aux} ~ 200 MW (cryoplant, pumps etc.)

How?

→Q_{eng} ~ 2!

DEMO:

→ Must reduce P_{aux} and P_{inj} NB: Off axis power (e.g. CD) has lower η

➔ advantage to Stellarators and Pulsed Tokamaks



A QCW Tokamak

Pulse No: 24806







Heat storage module 20m long, 850 tons cast iron Belgatom study, W. D'haeseleer et al 1994



- Generic issues
 - Undesired bifurcations; Plasma control under α heating
 - UFO; TAE
 - Fuel inventory
 - Choice of 1st wall material
 - Overall engineering Q (impact of control auxiliaries)
- Specific to advanced tokamak issues
 - 90% bootstrap, MHD, NTM
 - Stabilisation requires auxiliary power

 \rightarrow too much of it?



Where are we? LPO experience

• LHD: t ~ 54', P_{inj} ~ 490 kW, 1.6 MJ or 13', P_{inj} ~ 1.1 MW



• TRIAM-1M: t = 5h 16', 0.28 GJ (Nov 03)



Very good progress but not at reactor n and β !



- Large superconducting tokamak in operation
 - Cryo-magnetic system with trouble free operation for 20 years
 - All PFCs actively cooled
 - Powerful H/CD systems
- 6 min discharges (2003) with 1 GJ injected/exhausted energy
 - $P_{LHCD} = 3 MW$ - $I_p = 0.5 MA$ - $n_{e0} = 2.5 \ 10^{19} m^{-3}$

Missing n, β, bootstrap → more H&CD power required

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- Convective losses routinely handled by the TPL (3-5 MWm⁻²)
- Routine high power long duration discharges
- Studies of discharges in which
 - loop voltage is evanescent for duration much longer than current diffusion time
 - Stationary PFC surface temperature
- Testing key technologies in real Tokamak environment

Need longer CD power or bootstrap → New klystrons being procured

Non-linear behaviour @ Vloop=0

Oscillations of core electron temperature

TORE SUPRA

France

• Complex coupling between q-profile, MHD (double tearing, e-fishbones), transport and heat sources





Where are we? Non-inductive operation with RT control

Fully non-inductive regimes with large bootstrap over several resistive times achieved in many tokamaks

JT60-U (75%)

JET (30%-40%)





Impurity Issue

• Core impurity accumulation in negative magnetic shear configuration (Internal Transport Barrier)

e-heating shows control capability (JET, AUG)





- A virtuous circle seems to exist, related to the intimate link between current density profile and heat transport
- Only a partial solution so far: particle transport, rotation, edge, ..?

More modelling, theory, R&D and longer pulses needed

ERONOS

Will α heating preserve the loop?



Bécoulet EPS 2008

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Technology Issues

- Supra conductors
 - High T_c ? \rightarrow Higher engineering Q
- Erosion/redeposition/T inventory
 - Choice of material
 - Carbon, Tungsten etc..
- Structural longevity
 - Radiation resistant material
 - Disruption (Tokamaks)



Progress in superconducting coil R&D

One of 1st SC magnets (courtesy Oxford Instruments)

(1962)



Winding machine of helical coils for LHD (JPN, 1998)



Non-planar winding for W7-X (EU)



And now EAST and KSTAR



The Nb₃Sn conductors of the ITER coils



CSMC 51 mm x 51 mmm40 kA 13 T

TFMC Φ 40,7mm, 80 kA, 9.7 T







 Long term constant retention rate observed (~ 50% injected flux) from particle balance of long discharges



Where does it go?

D inventory: the 5hour marathon TS project

• Three steps:

- Dedicated long discharge campaign in D
- Dismantling one sector of TPL (20°) for samples extraction
- Analysis of samples
- The campaign
 - 160 long discharges (1 min < duration < 2 min)</p>
 - Total of 5 hours of plasma
 - no wall conditioning during the entire period
 - **10** g (3 10²⁴ D) retained of **19.3** g (5.8 10²⁴ D) injected
 - $-\sim 25~g$ of C redeposited on TPL

Objective:

Explore the erosion /re-deposition issue

B. Saoutic et al. SOFT 2008

J. Jacquinot, ITC 18 Toki 2008



- No sign of saturation
- First analysis conducted on 10 samples situated in 3 zones of interest: xerrorsion, thin & thick deposition zones
- Includes analysis of creter-tile gap depresionaries (LH = 1.6-1.8 MW Wall inventory Extrapolation to whole TPL :





Need for worldwide cooperation

- Preparing Long Pulse Operation of fusion devices is of strategic importance for the future of Magnetic Fusion
 - We are far from a demonstration of CW/QCW operation at reactor normalised parameters

> Slow equilibrium drifts, first wall material, Elm's, α confinement (wall loading by α losses)

- Long Pulse Steady-State developments deserve a worldwide coordinated effort:
 - Joint Experiments (short & long pulse, Tokamaks & Stellarators)
 - Theory / Simulation (First principle; Integrated & Real Time)
 - Technology R&D programme



In summary

Paris, 21 nov 2006

CW/QCW operation : A major challenge for fusion Broad topics: Technology and physics A world wide coordinated effort seems indispensable