

# Searching for a Flux Expansion Divertor in TJ-II.

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- Introduction and Motivation.
- The chosen configuration in TJ-II.
- ISDEP code and the plasma conditions.
- Results.
- Conclusions and future work.



**Divertor Concepts** 



Power exhaust and density control are mandatory for a stellarator-reactor (R. König et al. PPCF,2002). Concepts:

- X- point (Tokamaks).
- Helical Divertor.
- Island Divertor.
- Flux Expansion Divertor
- The goals:
  - Concentrate plasma-wall interaction in favourable zones (with plates and pumping).
  - Diminish the incoming fluxes on the plates (ad hoc magnetic configuration; increasing the plate size).
  - Hinder the neutrals to enter the plasma: Long path or steep gradient pressure profile.





#### **Helical Divertor**

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#### LHD (N. Oyabu et al. nf 1994)<sup>(0 = 18°</sup>





X-point







Figure 3. Plasma cross-section. The cross-sections on the right were produced on JET with the X-point just at the limit of the plasma wall. In these conditions, H-modes have been obtained. On the right, the plasma cross-section shows the approximate position of an X-point limiter in red; this plasma cross-section corresponds to an ITER cross-section shape, in fact the plasma shown in figure 2. P. Rebut. PPCF 2006



**Flux Expansion Divertor: NCSX** 





- Most flux expansion at 0<sup>o</sup> best for divertor plates
- Green field lines launched at inner midplane
- Red field lines launched at outer midplane







- Helical Divertor and Island Divertor require robust magnetic topology, almost unchanged during plasma operation.
- Those concepts are not suitable for:
  - -Configurations based on bootstrap current (NCSX, QPS).
  - -Equilibrium topology strongly dependent on beta.
  - -Flexible devices that can vary the rotational transport profiles (TJ-II).
- Flux Expansion Divertor could be the solution.





- Large flux expansion configurations do exist.
- No ergodic zones appear in TJ-II. Look for a strategy for creating them.
- The inner part of the configuration should be unaffected.
- Lithium evaporation onto these target plates?
- Pump behind plates?

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## **The Magnetic Configuration**



- Poincaré plots of the magnetic surfaces (rotated  $4\phi$  to be compared).
- Maximum flux expansion around  $\phi = \pi/4$  and  $\theta = \pi/2$ . Although wide  $\phi$ -range envisaged.

![](_page_9_Picture_4.jpeg)

![](_page_9_Picture_5.jpeg)

# **ISDEP code: Examples of 3D orbits**

![](_page_10_Picture_1.jpeg)

![](_page_10_Figure_2.jpeg)

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• Following particle trajectories rather than field lines (10<sup>5</sup>-10<sup>6</sup> particles). Grid computing inside EGEE. (F. Castejón et al. PPCF 2007)

- Guiding centre approximation.
- i-i & i-e Coulomb collisions.
- **Considering electrostatic** potential.
- No assumptions on orbit size or diffusive transport #10

## Measured Profiles (used in the simulatiuon)

- Two regimes: low density ECRH plasma and high density NBI plasma.
- Simulations valid for a wide range of parameters.
- n<sub>e</sub> and T<sub>e</sub>, from Thomson Scattering.
- T<sub>i</sub> from CX-NPA.
- Potential from HIBP.

![](_page_11_Figure_6.jpeg)

![](_page_11_Figure_7.jpeg)

![](_page_11_Picture_8.jpeg)

### The Map of Losses

![](_page_12_Picture_1.jpeg)

![](_page_12_Figure_2.jpeg)

- Maximum plasmawall interaction on the groove.
- PWI close to the plasma bulk.
- Up-down asymmetric flux.

![](_page_12_Figure_6.jpeg)

**Obtaining the Map of Fluxes** 

![](_page_13_Picture_1.jpeg)

- We locate  $N_{\phi} \ge N_{\theta} = 4 \ge 32 = 128$  plates in each period.
- A single plate (ρ,i,j):

$$\rho > \rho_0; \quad \frac{2\pi}{N_{\phi}}i < \phi < \frac{2\pi}{N_{\phi}}(i+1); \quad \frac{2\pi}{N_{\theta}}j < \theta < \frac{2\pi}{N_{\theta}}(j+1)$$

- $\theta$  rotated -4 $\phi$  with respect to the horizontal plane (groove-magnetic axis line:  $\theta=0$ ).
- The toroidal range of  $\theta = -\pi/2$  :

![](_page_13_Picture_7.jpeg)

Flux Map: ECRH

![](_page_14_Picture_1.jpeg)

![](_page_14_Figure_2.jpeg)

- The plates are independent.
- The shadow effect is not considered in this calculation.
- Clear poloidal and toroidal structure.
- Small changes for ρ>1.

Flux Map: NBI

![](_page_15_Picture_1.jpeg)

![](_page_15_Figure_2.jpeg)

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## Flux on the chamber: ECRH

![](_page_16_Picture_1.jpeg)

![](_page_16_Figure_2.jpeg)

![](_page_17_Picture_1.jpeg)

![](_page_17_Figure_2.jpeg)

![](_page_18_Picture_1.jpeg)

- A promising configuration for Flux Expansion Divertor has been found in TJ-II.
- ISDEP code has been used to calculate the maps of the fluxes in ECRH and NBI plasmas.
- The flux map shows a strong poloidal structure: It is possible to locate divertor plates to suppress a large fraction of the total flux onto the wall.
- The flux on the groove can be strongly reduced, which is critical to diminish the plasma-wall interaction in TJ-II.

![](_page_18_Picture_6.jpeg)

![](_page_19_Picture_1.jpeg)

- Refine the design of the plates.
- Explore the possibility of creating an ergodic layer outside the LCFS.
- Estimate the recycling & transport of neutrals.
- Consider the Li coating?
- Thinking of experiments.

![](_page_19_Picture_7.jpeg)

![](_page_20_Picture_0.jpeg)

![](_page_20_Picture_1.jpeg)

![](_page_21_Figure_0.jpeg)

![](_page_21_Picture_1.jpeg)

![](_page_22_Picture_1.jpeg)

![](_page_22_Figure_2.jpeg)

![](_page_22_Picture_3.jpeg)

![](_page_23_Figure_0.jpeg)

![](_page_23_Picture_1.jpeg)