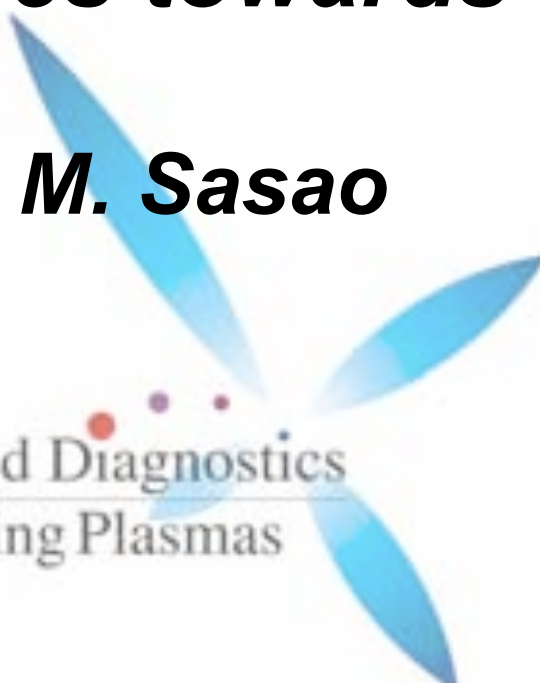


# ***Innovation on ITER neutron diagnostics towards DEMO***

***M. Sasao***



Advanced Diagnostics  
for Burning Plasmas



# Out lines

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- Introduction
- Neutron diagnostics on ITER
- Fusion output measurement and the calibration strategy for it
- Neutron diagnostics for physics understandings
- Summary

This presentation was prepared on the basis of discussion among the ITPA-diagnostics Neutron working group.

# Introduction

## - Fusion Reaction Rates



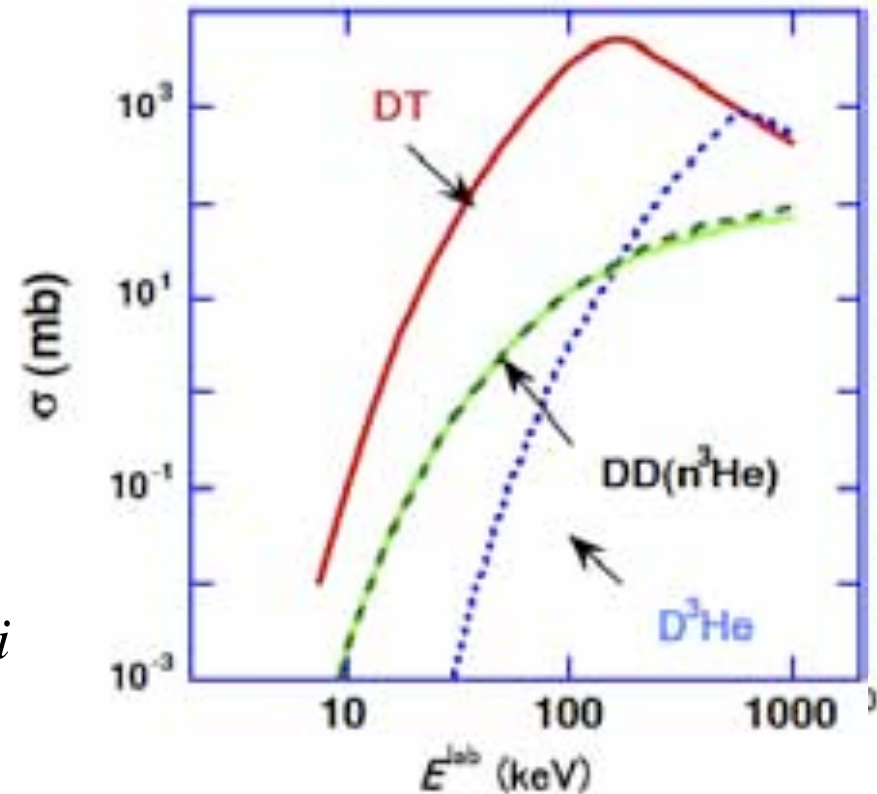
Three terms constitute an actual reaction rate in a plasma of an unit volume, the thermo-nuclear term, the beam thermal term and the beam-beam term.

$$Y_{\text{total}} = Y_{\text{th}} + Y_{\text{b-th}} + Y_{\text{b-b}}$$

$$\begin{cases} Y_{th} = n_j n_i \langle \sigma v \rangle_{Ti} & j \neq i \\ Y_{th} = \frac{1}{2} n_i^2 \langle \sigma v \rangle_{Ti} & j = i \end{cases}$$

$$Y_{\text{b-th}} = n_b n_i \langle \sigma v \rangle_{\text{b-th}}$$

$$Y_{\text{b-b}} = \frac{1}{2} n_b^2 \langle \sigma v \rangle_{\text{b-b}}$$



Fusion cross sections as a function of the energy of the reacting particles(a) and fusion reactivities for Maxwellian ion distributions as a function of  $T_i$  (b).

# Introduction

## - Role of neutron measurement -present



• In many magnetic-confinement fusion devices to date, where tritium is not introduced, but the plasma contains energetic deuterons, the dominate term is the beam-thermal DD term,  $Y_{b-t}$ .

• The measurement of fusion reactivities in the present devices does provide the information of **energetic-ion behaviors**, not directly provide the THERMO-NUCLEAR fusion output.

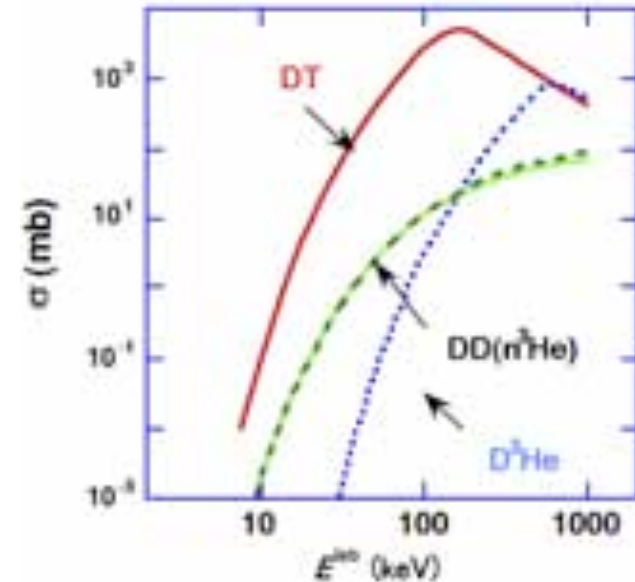
• Tritons produced through



undergo



and confinement of energetic-ion can be simulated (triton burn-up)



$$Y_{b-th} = n_b n_i \langle \sigma v \rangle_{b-th}$$

# Introduction

## - Role of neutron measurement -ITER



• In ITER, both thermal,  $Y_{th}$ , and the beam-thermal term,  $Y_{b-t}$  contribute, but the thermal reaction is dominant in self heating phase.

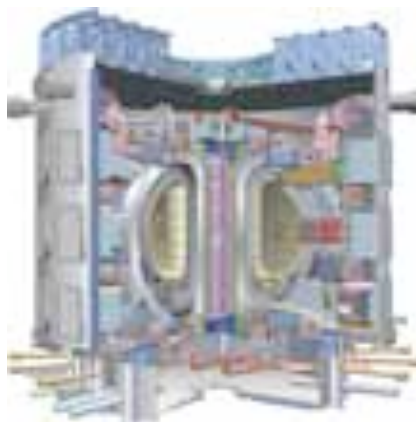
$Y_{b-t}$ : External beam Heating phase

30MW NBI,  $10^{20}$  Plasma  $\sim 3 \times 10^{16}$  n/m<sup>3</sup>

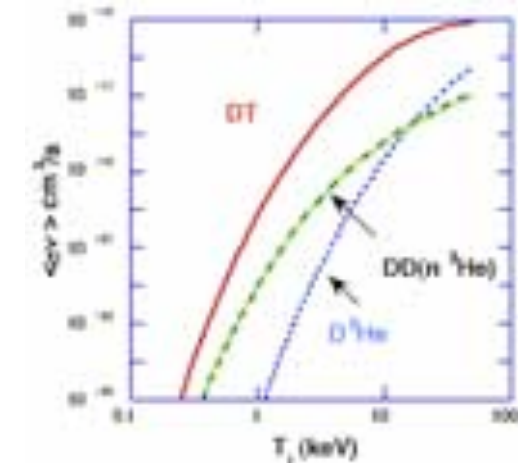
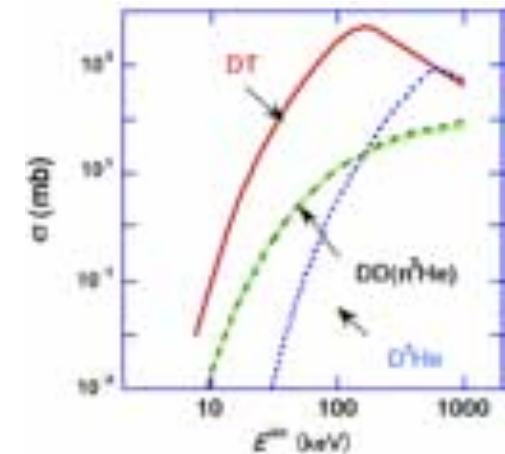
$Y_{th}$ :

Ti=10keV,  $10^{20}$  Plasma  $\sim 3 \times 10^{17}$  n/m<sup>3</sup>

Ti=20keV,  $10^{20}$  Plasma  $\sim 10^{18}$  n/m<sup>3</sup>



major radius	6.2	m
minor radius	2.0	m
volume	840	m <sup>3</sup>
plasma current	15.0	MA
on-axis toroidal field	5.3	T
fusion power	500	MW
burn flat top	>400	s
energy multiplication	>10	



# Introduction

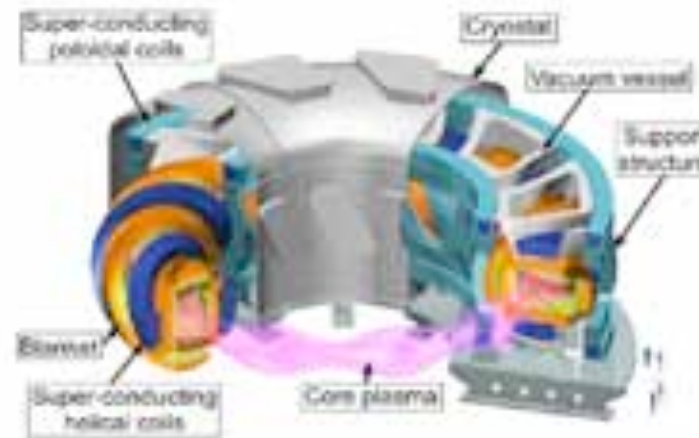
## - Role of neutron measurement DEMO



Only the thermal reaction contributes to the neutron emission.

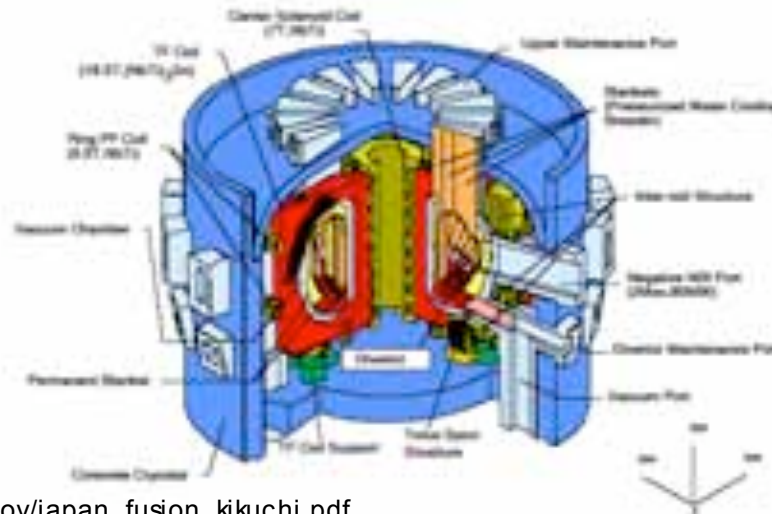
$$P_{\text{output}} \sim 2\sim 3 \text{ GW}$$

$$P_{\text{output}}/P_{\text{ext}} \sim 50\sim\infty$$



Nucl. Fusion **47** (2007) 1411–1417

Figure 1. Reactor view of ITER 7m helical reactor



		Design Value
Plasma Current	$I_p$	12MA
Toroidal Field Coil	$B_t$	9T
Major Radius	$R$	7m
Aspect Ratio	$A$	4.1
Elongation	$\bar{\Delta}_0$	1.85
Normalized Beta	$\bar{\Delta}_0$	3.5
Fusion Output	$P_f$	30GW
Current Drive Power	$P_{CD}$	60MW
Net Electric Output Power	$P_e$	1.08GW
Fusion Gain	$Q$	50
Averaged Neutron Wall Load	$P_{\text{neut}}$	3MW/m <sup>2</sup>

[http://fire.pppl.gov/japan\\_fusion\\_kikuchi.pdf](http://fire.pppl.gov/japan_fusion_kikuchi.pdf)



# Introduction

## - Information from Neutron Diagnostics on ITER

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Fusion Output (Total reaction rate), Neutron fluence on the First Wall

Emissivity Profile, Profile changes

Energetic Particle Behavior, redistribution

Ion temperature, Ion temperature profile, Profile change

Energy Spread

Energetic Particle Behavior, Slowing down process

Ion temperature

Super Energetic Neutrons

Alpha Knock-on

$Y_{DT}/Y_{DD}$  ratio

$n_T/n_D$  ratio

# Neutron Diagnostics on ITER

## - systems

---



Fusion Output (Total reaction rate), Neutron fluence on the First Wall

Neutron Flux monitors, activation systems

Emissivity Profile, Profile changes

Radial Neutron Camera, Vertical Neutron Camera

Energy Spread

Compact neutron spectrometers in Radial Neutron Camera,

Compact neutron spectrometers in Vertical Neutron Camera

Super Energetic Neutrons

High Resolution Neutron Spectrometer

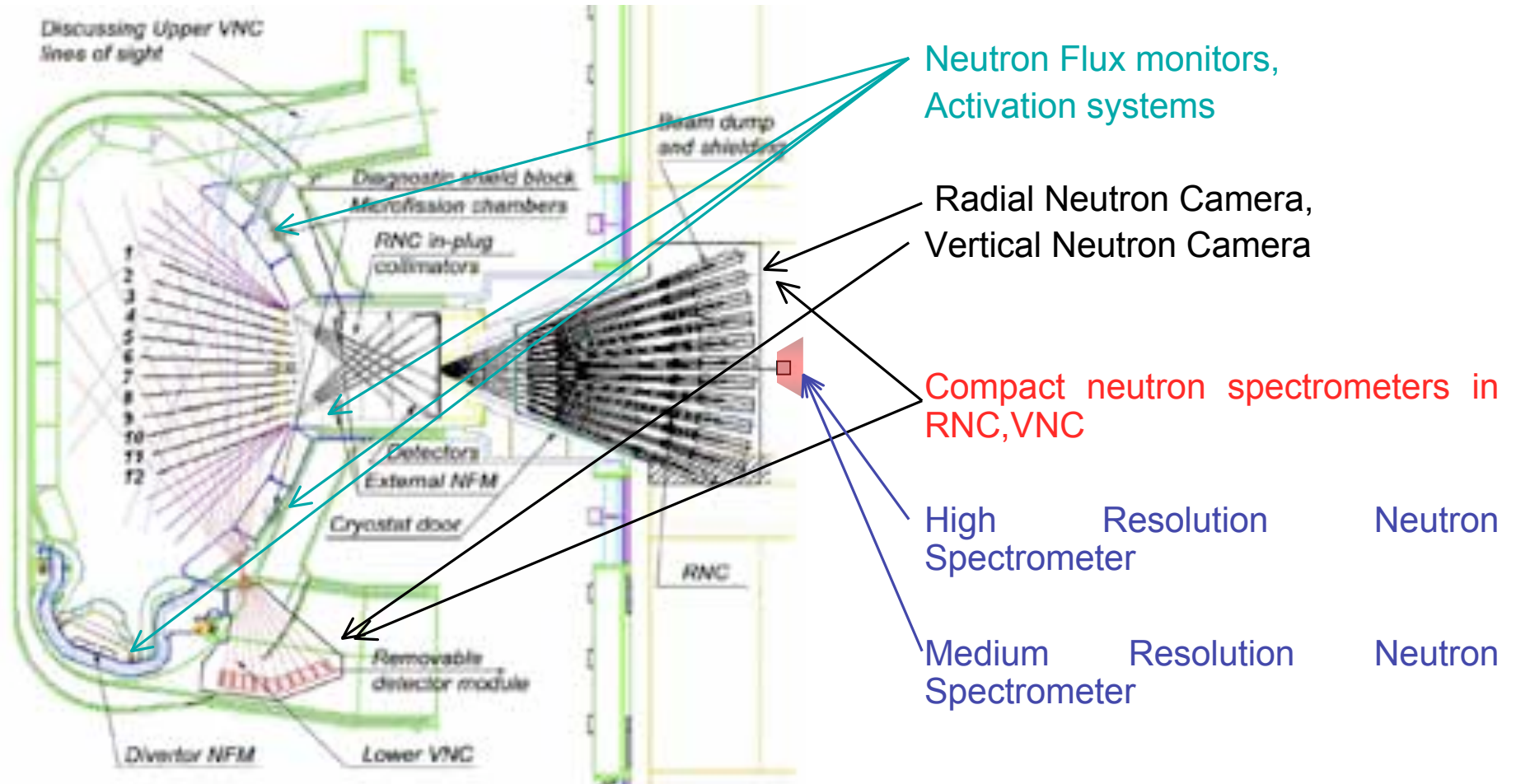
$Y_{DT}/Y_{DD}$  ratio

Medium Resolution Neutron Spectrometer



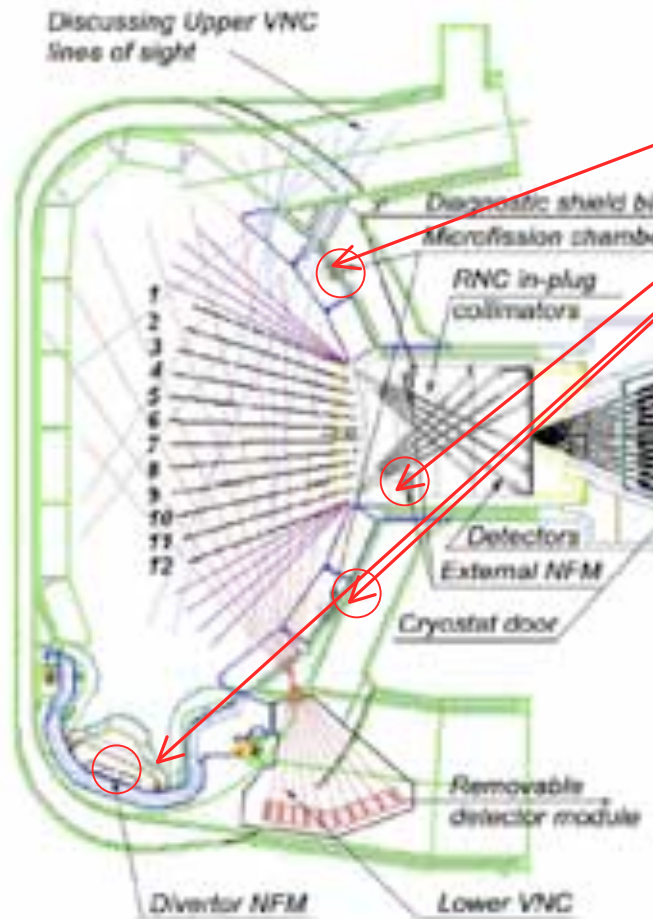
# Neutron Diagnostics on ITER

## - systems



# Fusion output measurement

- reliability and accuracy

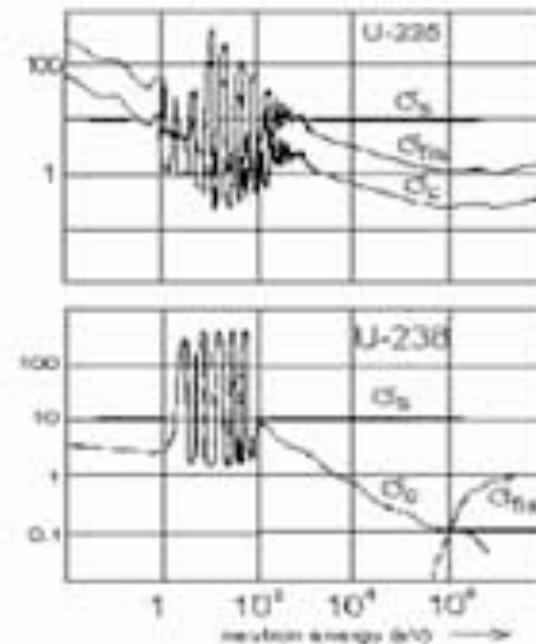


## Neutron Flux monitors

Detectors are sensitive to low energy neutrons ( $1/v$  dependence).

Blanket modules and shield modules function as a neutron energy degrader and a moderator.

**10%  
accuracy is  
required**



<http://www.ricin.com/nuke/gifs/cross.gif>



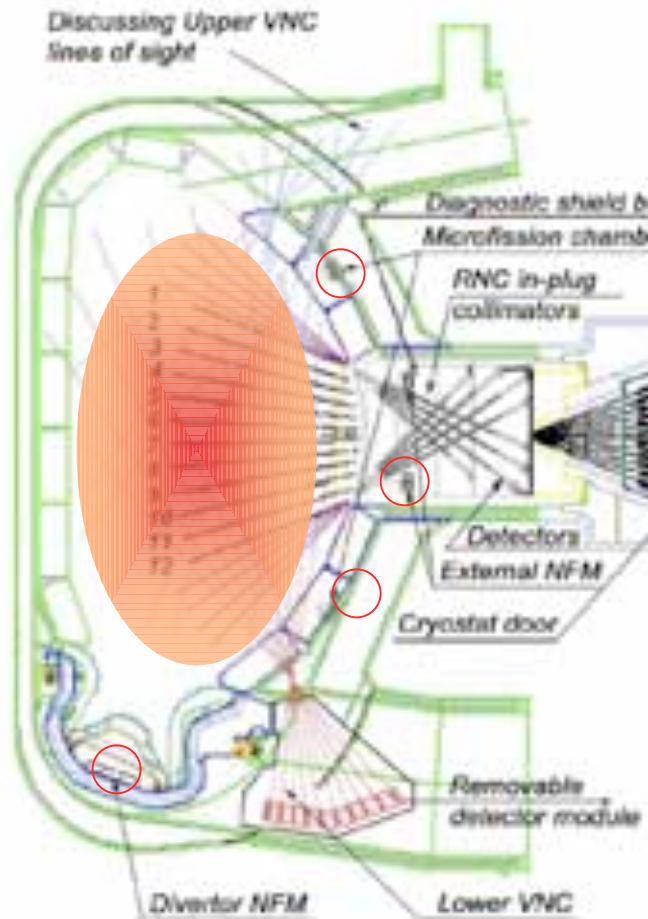
# Out lines

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- Introduction
- **Neutron diagnostics on ITER**
- Fusion output measurement and the calibration strategy for it
- Neutron diagnostics for physics understandings
- Summary

# Innovation on Fusion output measurement

## - an extended source



Neutron source is a volume source, extended into a donut shape.

$$C_n(\vec{X}_D) = \int Y(\vec{r}) \eta(\vec{r}, \vec{X}_D) d\vec{r}$$

$$C_n(\vec{X}_D) \quad Y(\vec{r}) \quad \eta(\vec{r}, \vec{X}_D)$$

They are energy dependent.

Calibration using a radio isotope ( $^{252}\text{Cf}$ ), DT/DD generators is needed.

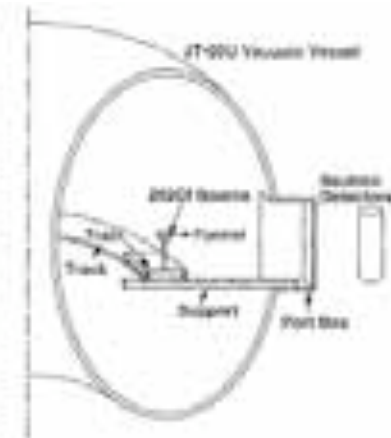
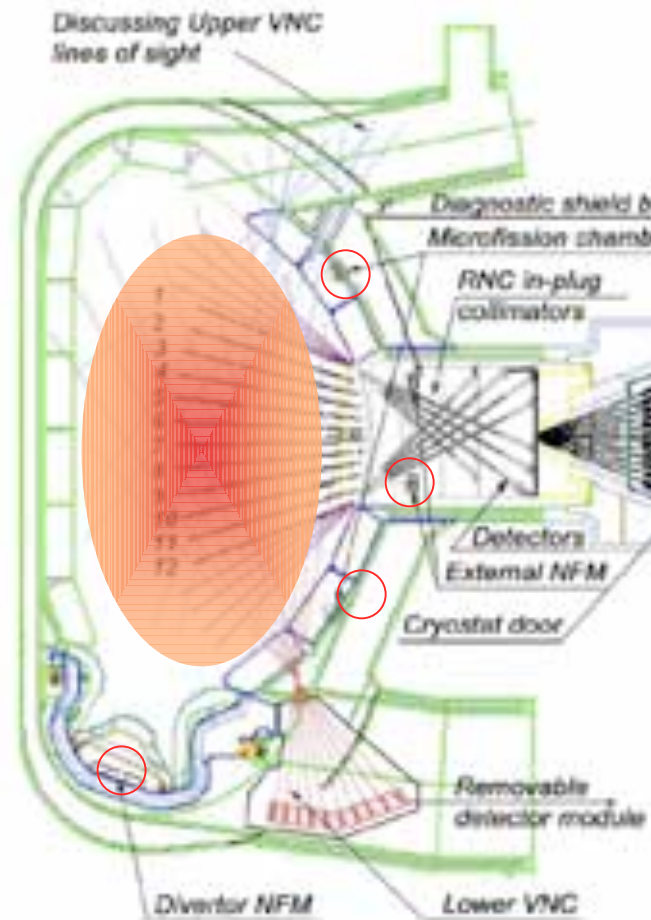


FIG. 4. Schematic of the neutron transfer system in the recessed vessel of JT-60U.

T. Nishitani  
Rev. Sci. Instrum. 63, 5270 (1992).

# Innovation on Fusion output measurement

- should be supported by



## Calibration

MCNP calculation is needed because of the difference in the neutron energy distribution / self-shadow effect.

Connection of measurement by many detectors/electronics

## Activation systems

foils

water circulation - DEMO relevant  
coolant temperature change  
- DEMO relevant

## Profile measurement

Profile changes

$Y_n(x)$  is not a function of  $\Psi$ .

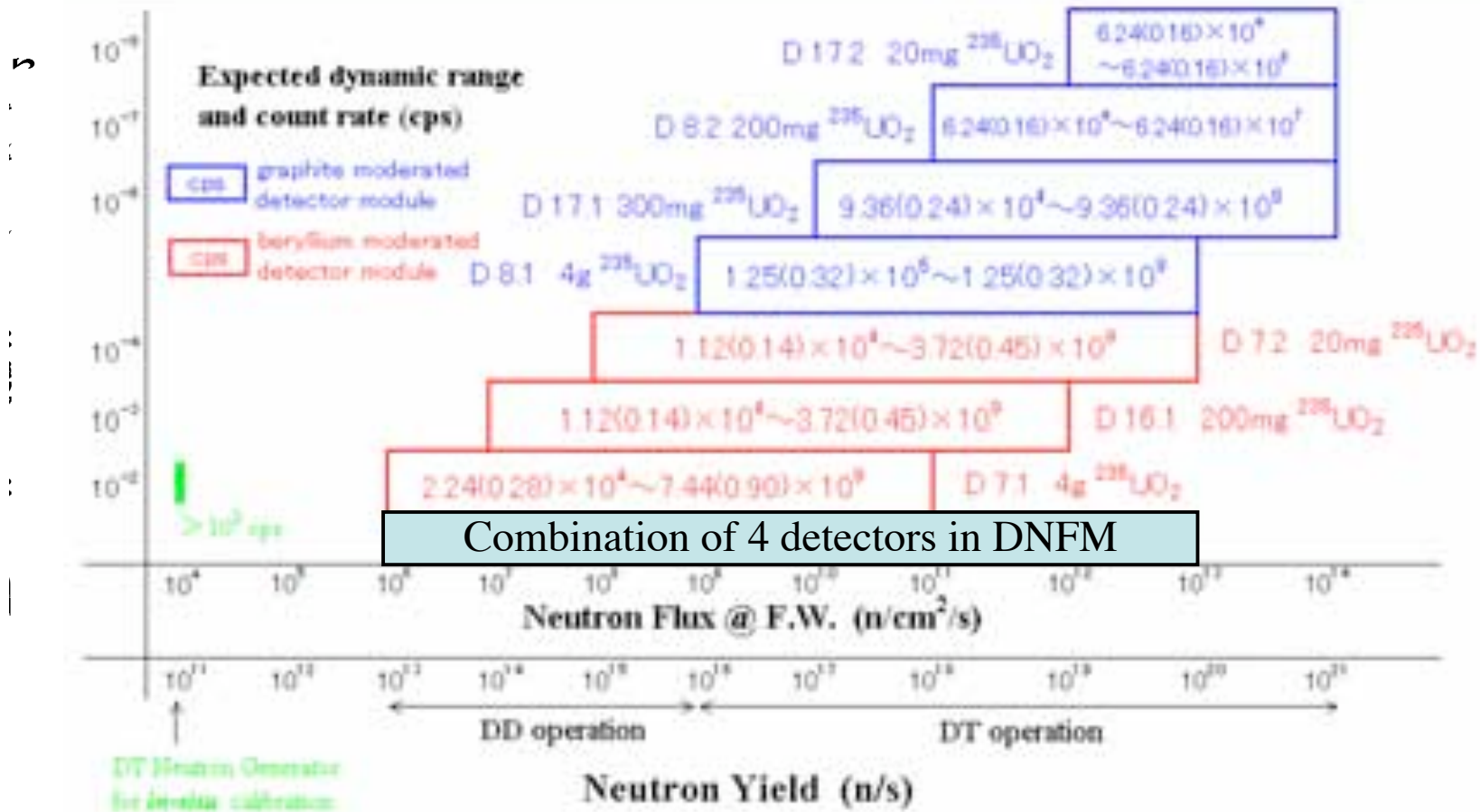


# Innovation on Fusion output measurement



- The dynamic range over 12 orders is required

## Combination of 2 MCF's

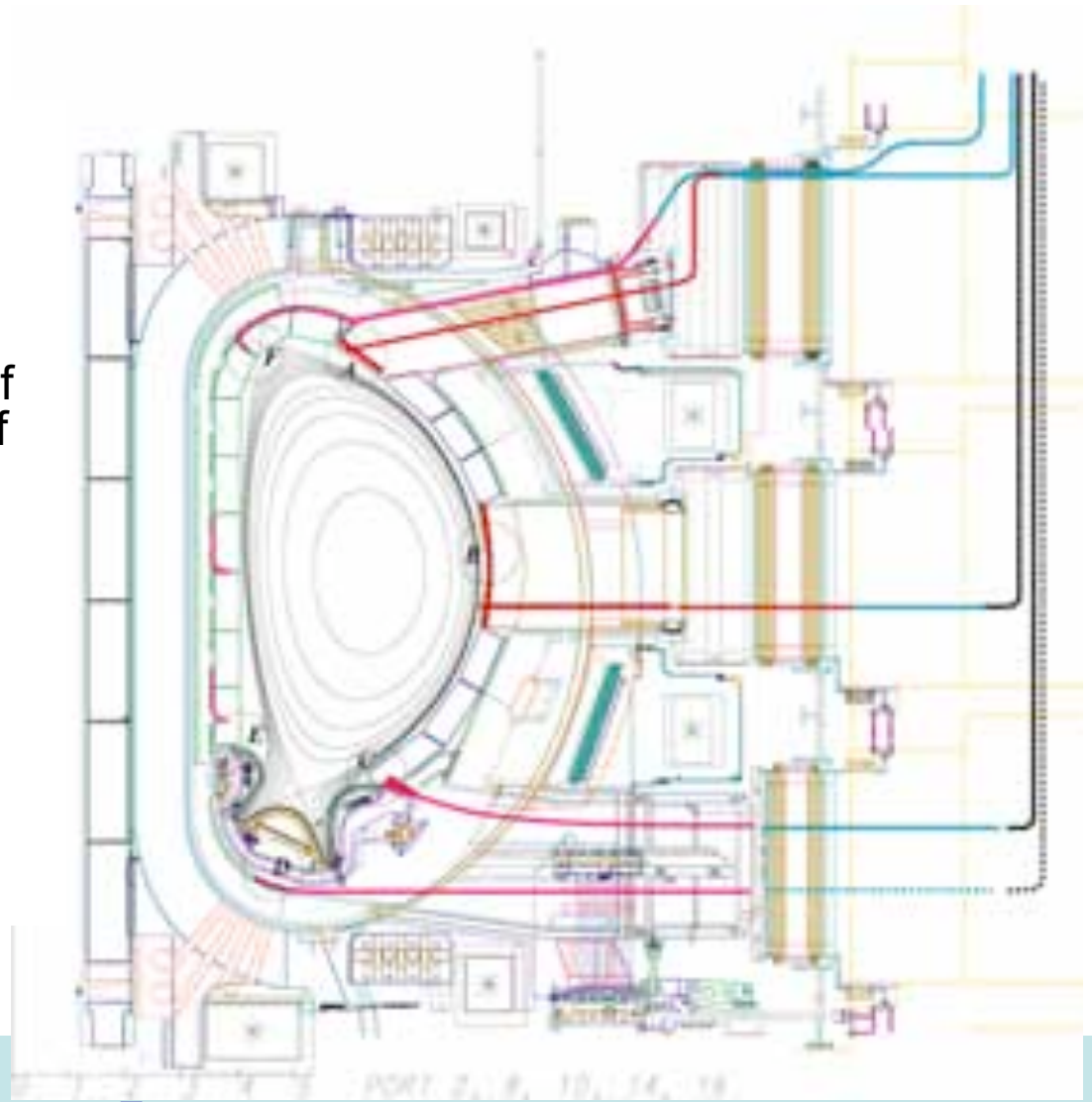


# Foil activation

Irradiation Station & Transfer Line are under being designed.

Several irradiation positions are needed to guarantee the accuracy against the change of the plasma axis position, that of the neutron emission, etc..

- A Upper port plug
- B Equatorial port plug
- C Lower outboard VV wall
- D Under Diverter Dome
- E Lower inboard VV
- F Upper inboard VV wall,
- G Center inboard VV



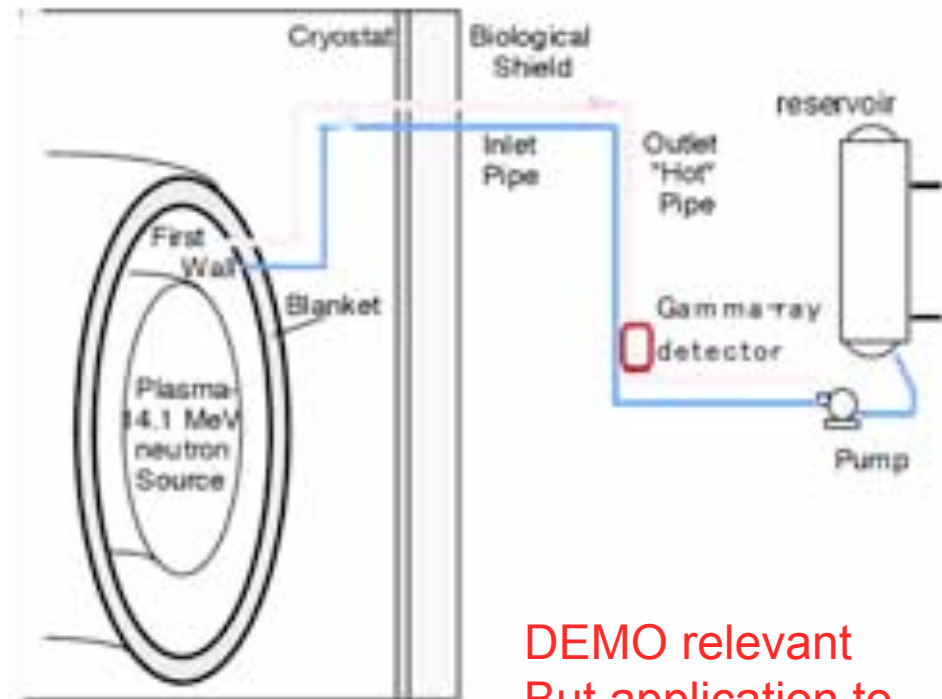
# Water Activation Systems

A neutron activation system with flowing water using the  $^{16}\text{O}(n,p)^{16}\text{N}$  reaction

6.13 MeV gamma rays  
the temporal resolution would be less than the ITER requirement of 100 ms including turbulent diffusion effects for the flow

velocity of 10 m/s. With a flow velocity of 10 m/s, this system can measure the fusion power from

50 kW to 1 GW of the ITER operation by using two gamma-ray detectors;



DEMO relevant  
But application to  
ITER is not decided  
yet

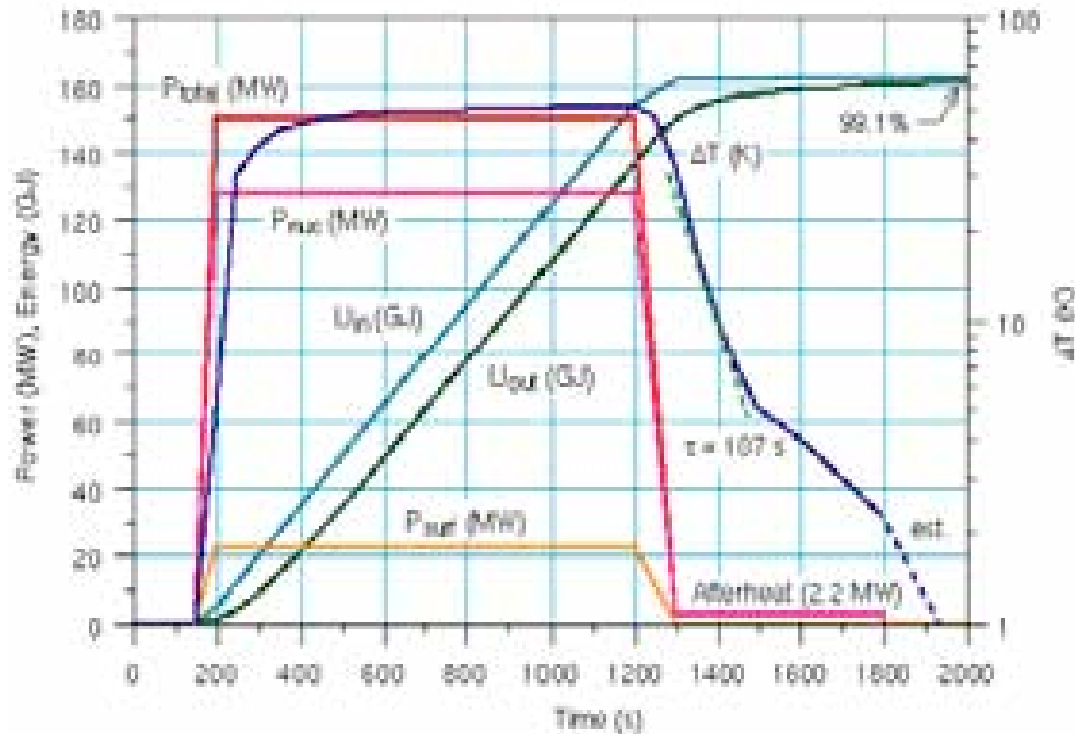
T. Nishitani  
Rev. Sci. Instrum. 74, 1735 (2003).



# Calorimetric measurements in ITER

J.C. Vallet & C.Portafaix

(EFDA-Euratom-CEA contract n° 03-1111)



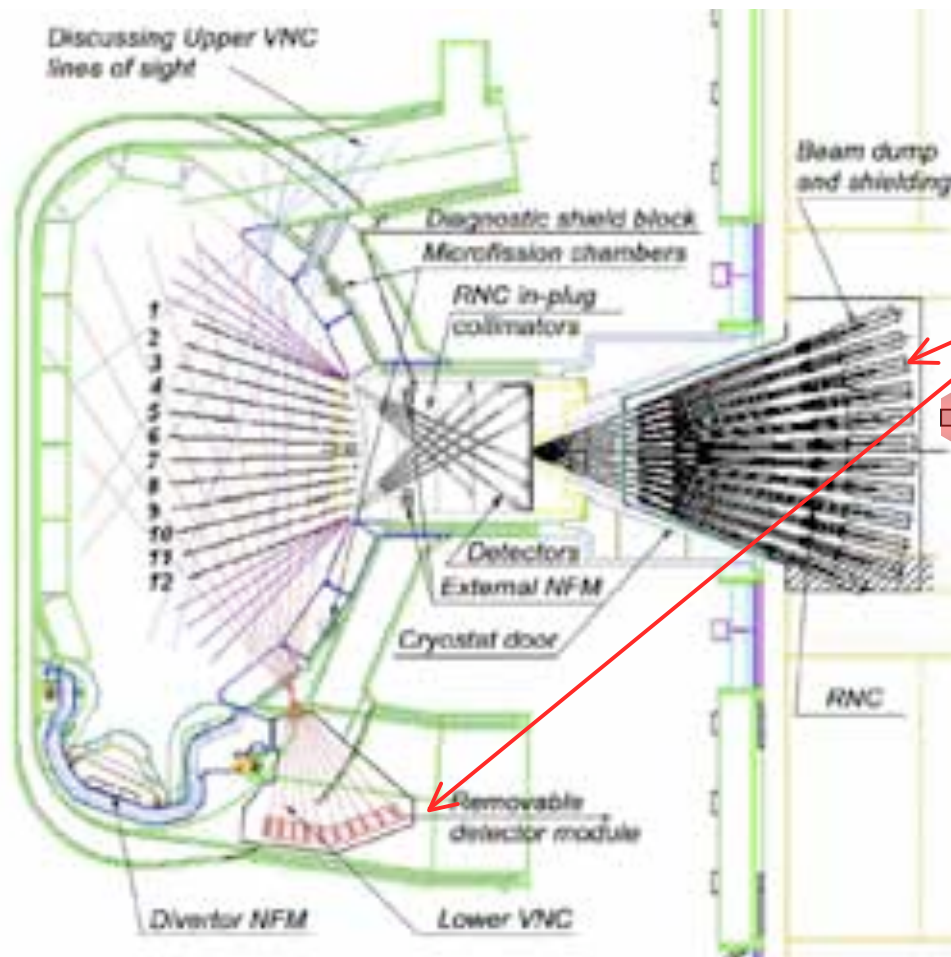
DEMO relevant  
But application to  
ITER is not decided  
yet

- The modelling provides  $t_C=30s$  for the FW and  $t_C=60s$  for the BS

Fig. 3. Waveforms for one PFW/IBB HTS circuit with reference 1.5 GW pulse  
**Fusion Power Measurement by Cooling Water Calorimetry**  
John Wesley; S 19 RI 8 97-07-10 F1



# Innovation on - Profile measurement systems



Accurate fusion output can not be obtained without neutron emission profile measurement.

Neutron Camera consist from

Multi channel collimator + detectors not sensitive to low energy neutrons and gamma's.

Some schintillator can discriminate neutrons from gamma's.

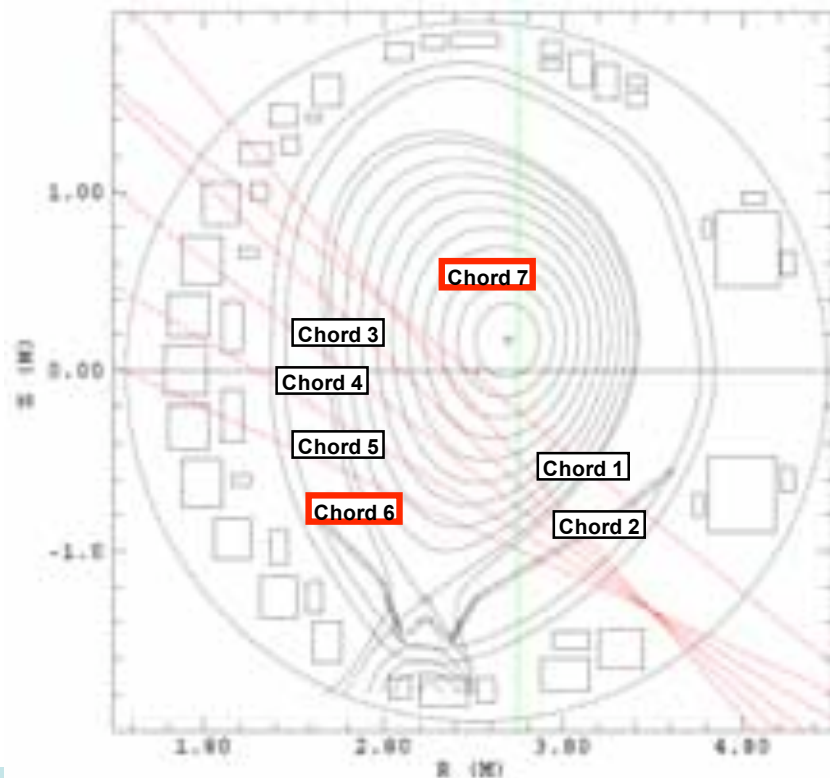
Fast electronics is needed.

- Radial Neutron Camera,  
Vertical Neutron Camera

# Neutron emission profile measurement in JT-60U

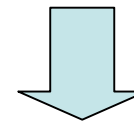


- Measure the line-integrated neutron emission
  - ⇒ infer energetic ion profile
- 7 channel collimator array viewing a poloidal cross section



- Detector
  - Stilben crystal neutron detector

***n-γ discrimination*** is necessary

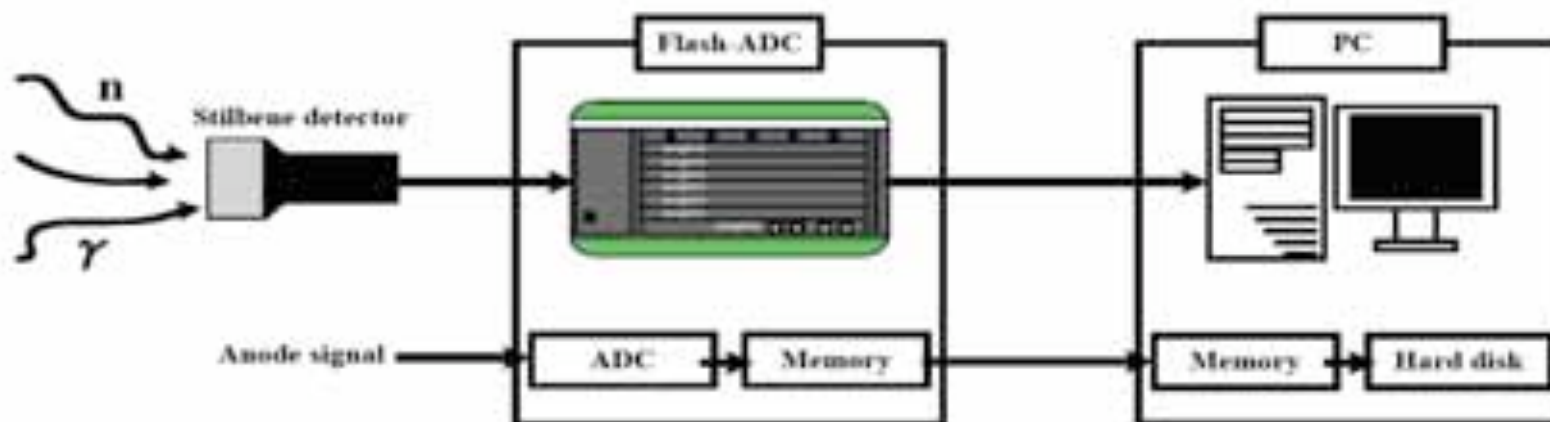


DSP-system and fast discrimination software were developed. (K. Ishii, ITC18-P238)



# Digital Signal Processors system

In the DSP system, output pulses from an anode of a photo multiplier tube (PMT) on a detector are recorded as continuous waveform using a fast flash analog-to-digital converter (flash ADC) [2.4.5].

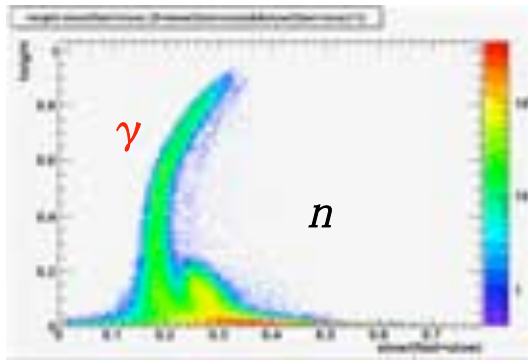


- [3] M. Ishikawa, T. Itoga, T. Okuji, et al., Rev. Sci. Instrum. **77**, 10E706 (2006).
- [5] T. Itoga, M. Ishikawa, M. Baba, et al., Radiat. Prot. Dosim. **126**, 380 (2007).
- [6] K. Shinohara, T. Okuji, M. Ishikawa, et al., Rev. Sci. Instrum. **79**, 10E509 (2008).

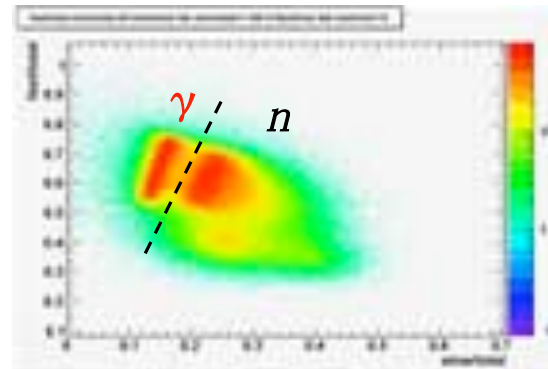
# Fast logics of Neutron-gamma discrimination



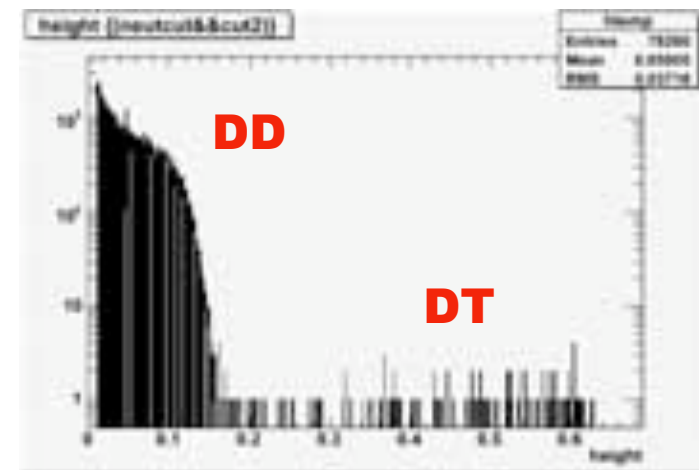
A new pulse-shape analysis method is developed and applied to all stilbene detectors. Figure shows a 2D map in fast-slow space normalized with total. Neutrons are discriminated from  $\gamma$ -rays clearly. This system enables routine measurement of DD neutron emission rate with time resolution of ms range, and also routine measurement of triton burn-up simultaneously.



Conventional  
n-gamma discrimination



New  
n-gamma discrimination



An example of Neutron spectra  
measured on one of channels, 50 ms.  
JT-60U

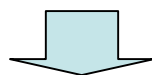
K. Ishii, ITC18-P238



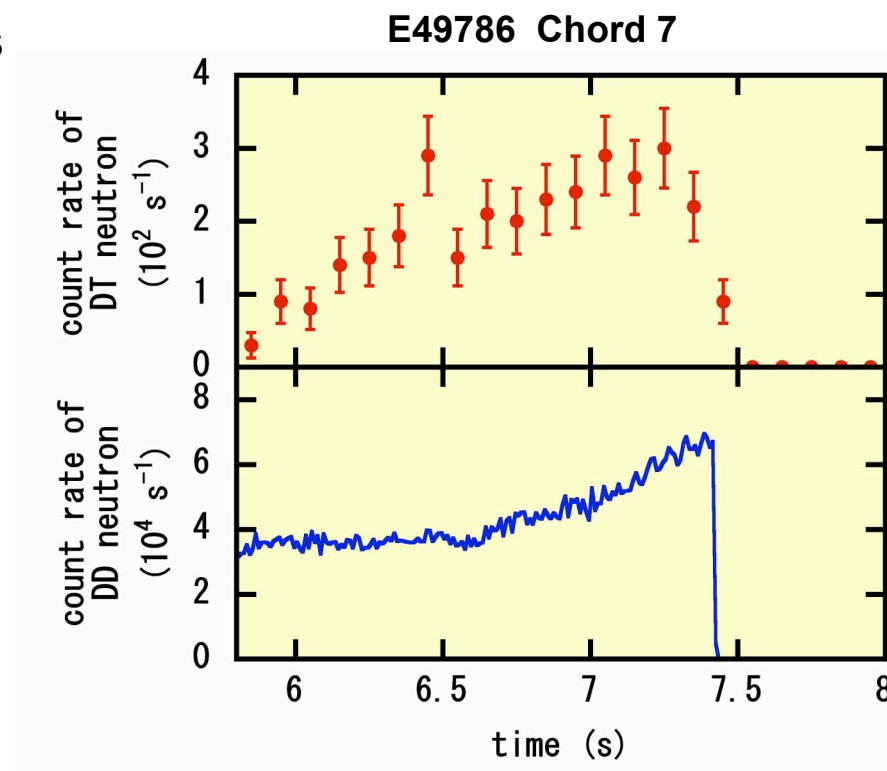
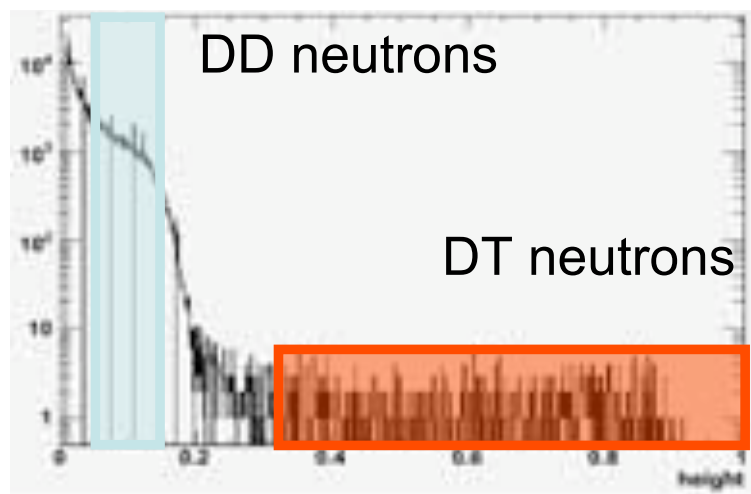
# Pulse Height Analysis

- Pulse height distribution is obtained to discriminate neutron events from  $g$ -ray events using both of conventional and new 2D map
- **DT neutrons is measured** as well as DD neutrons in the DD plasma

Define two regions for DD and DT neutrons



The time evolution of neutron emission rate is obtained



K. Ishii, ITC18-P238



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# Neutron diagnostics for physics understandings

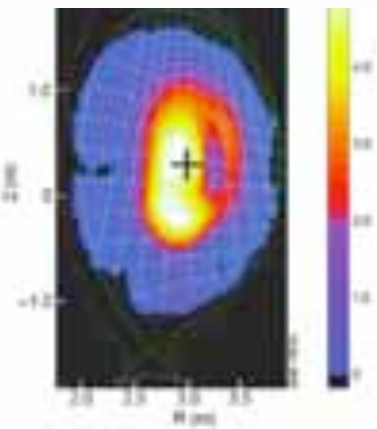
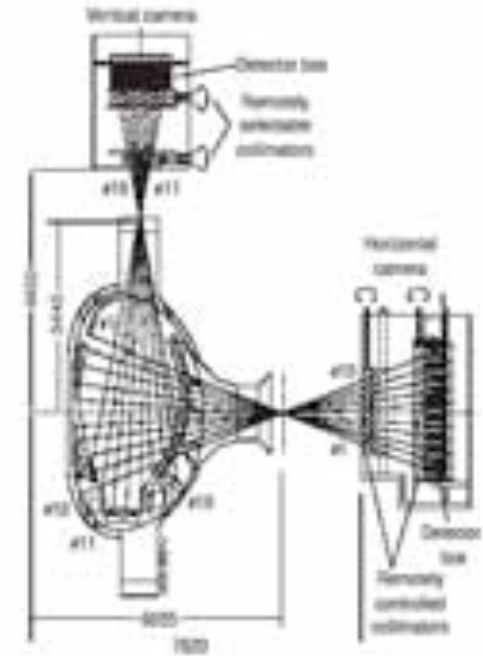


It was proven that measurement of  $Y_n$ , and neutron/gamma profile ( $Y_n(\psi)$ ), secondary nuclear reaction products (triton burn-up) are useful to understand physics on heating scenario's, and energetic particle resonant behaviors - fishbone instabilities, ALE, TAE, etc., on PBX, TFTR and DIII-D, TFTR, JET, JT-60U, etc. Information on re-distribution was obtained by the profile measurement.

These physics studies might be difficult on ITER, because the heat source is alpha particles.

Confined and lost alpha particle diagnostics are expected to take over the role.

However, information on bulk ion behaviors can be studied both by change of emission profiles and ion temperature profiles.







# Bulk ion behaviors and ion diffusivity have been studied on JET by neutron emissivity profiles

The local ion power balance of this region can be expressed as

$$\frac{dW_i}{dt} = Q_{ei} - Q_{cond} \approx \frac{3n_e(T_e - T_i)}{2\tau_{ei}} + \frac{1}{r} \frac{\partial}{\partial t} \left( rn_i(r) \chi_i(r) \frac{\partial T_i(r)}{\partial r} \right)$$

$$Y_n(\rho) = n_d(\rho) n_t(\rho) T_i^\beta(\rho)$$

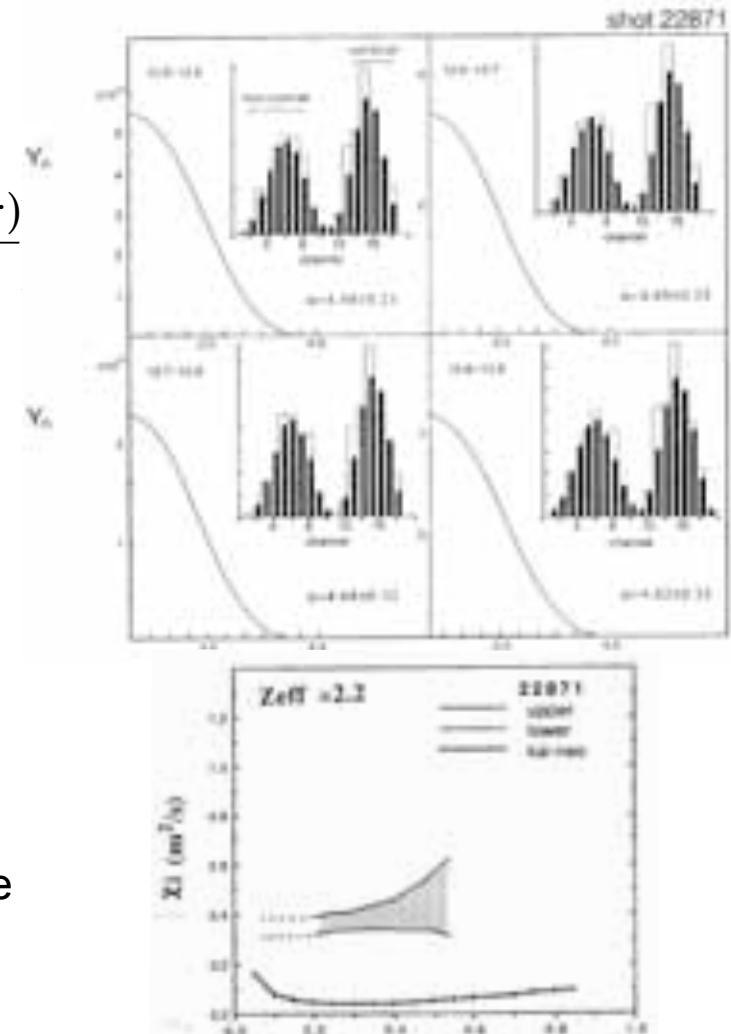
$$Y_n(\rho) = Y_0(\rho) (1 - \rho^2)^\alpha$$

This is the equation to obtain  $\chi_i(r)$ .

Absolute values of  $n_d(r)$ ,  $T_e(r)$ ,  $T_i(r)$ ,  $Y_n(r)$ 's are reducible.

Only the neutron emission decay constant and the profile factor of  $T_i(r)$ ,  $\alpha$  are main factors to determine the  $\chi_i(r)$ .

M. Sasao, et al., Plasma Physics and Control.Fusion, 36(1994)p1-8



# Alpha Knock-on effect on the neutron Spectrum

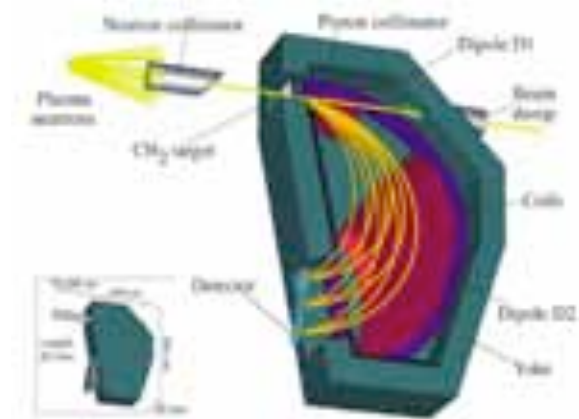
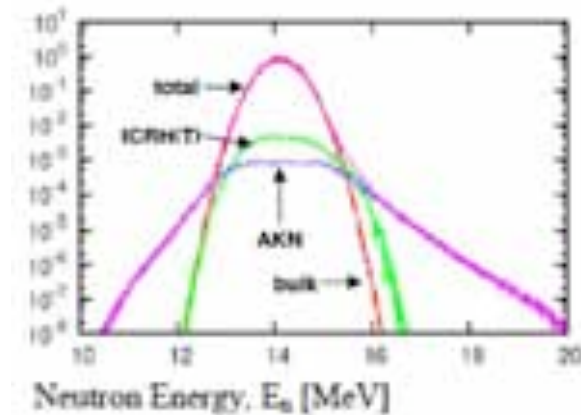


The population of fast confined  $\alpha$ -particles can give rise to alpha knock-on neutron (AKN) emission in DT plasmas in the high energy tail of the spectrum, produced by supra-thermal knock on ions.

A large magnetic spectrometer

Was developed for the measurement on JET-DT campaign.

M. Sasao, et al., Fusion Technology and Science, Special Issue on diagnostics, Chapter 9 (2008, Feb.)

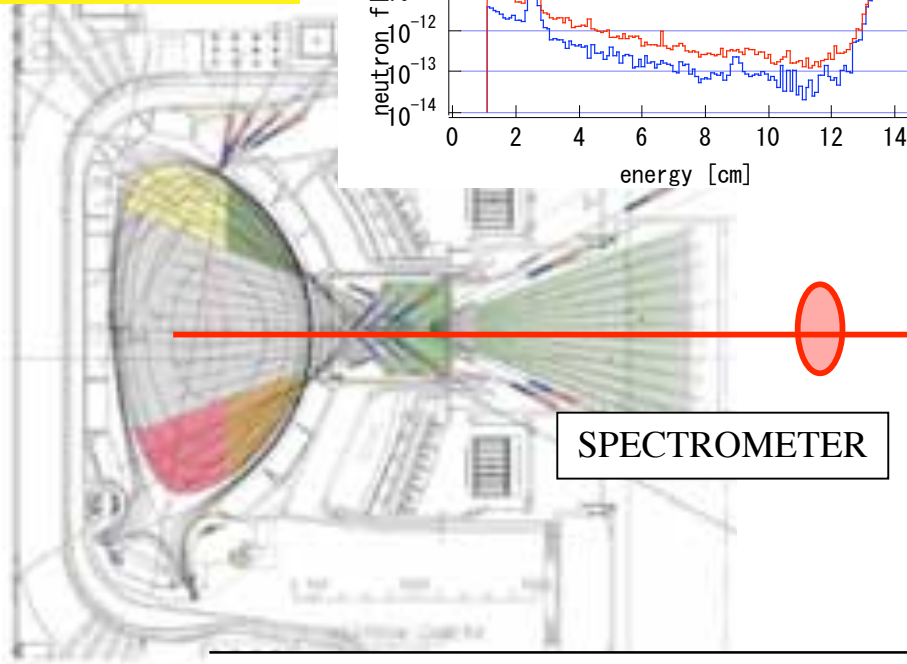
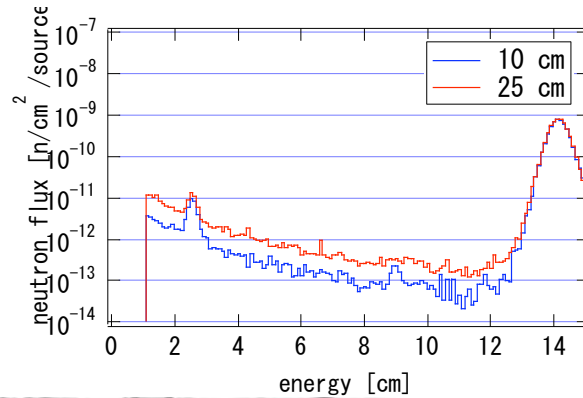


# TOF spectrometer for the ITER - DD/DT neutron ratio measurement - high efficiency, high cps

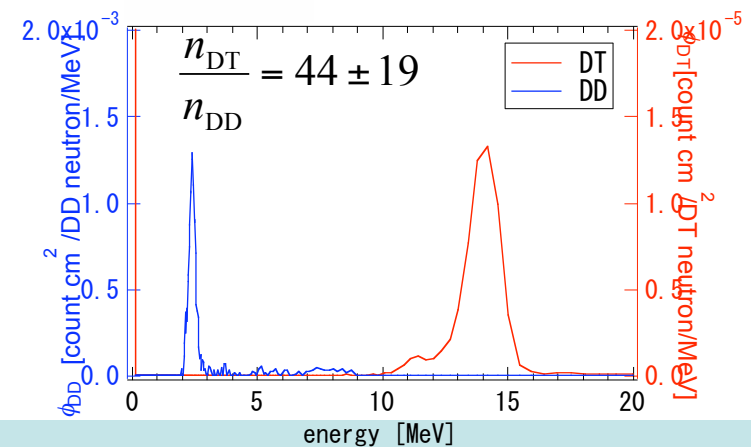
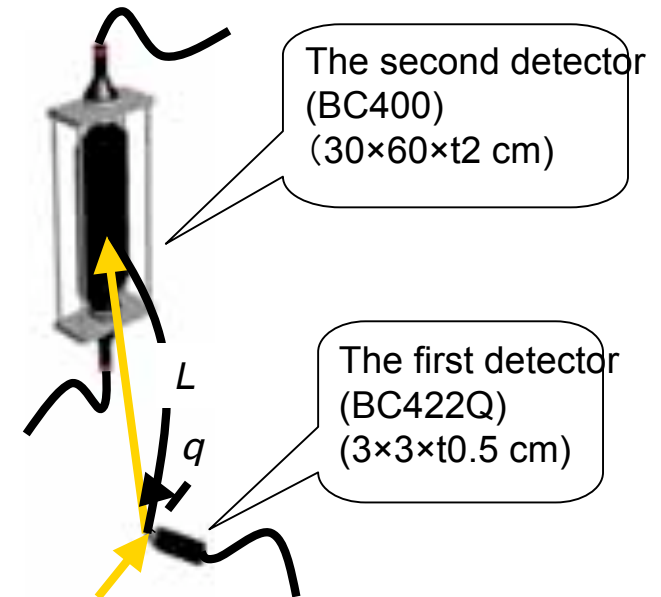


$$S_{DT} = n_D n_T \langle \sigma v \rangle_{DT}$$

$$S_{DD} = \frac{1}{2} n_D n_D \langle \sigma v \rangle_{DD}$$



[count cm <sup>2</sup> /neutron]	efficiency (resolution)
DT	$3.5 \times 10^{-5}$ (DE/E = 10.6 %)
DD	$3.9 \times 10^{-4}$ (DE/E = 11.3%)



# Summary



- Neutron measurement is one of the major diagnostic methods on ITER, and it will be one of few plasma measurement tools on DEMO. Its principal role on ITER is to allow evaluation of fusion output, indicating how close the ITER plasma approaches the ultimate goal of a self-sustained nuclear fusion reactor.
- Accuracy (10%) and reliability is demanded.
- Calibration experiment using ratio-isotopes, DD/DT neutron generators is needed, combined with MCNP calculation, foil-activations, and profile measurement.
- Connection of several detectors of different sensitivity is needed.
- In addition, neutron diagnostics supply a variety of both spatially and time-resolved information to facilitate our understanding of physics, especially with regard to the behaviours of energetic particle (heating phase), the ion temperature, the fuel isotope ratio ( $n_T/n_D$ ),  $\alpha$ -particle confinement, and so on.
- The time-resolved neutron emission profile is essential to study the ion transport of ITER plasma.